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Addressing Key Science and Technology Issues for IFE Chambers, Target Fabrication and Target Injection

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Abstract. Significant progress has been made in addressing critical issues for high repetition rate chambers, target fabrication and injection for inertial fusion energy (IFE) for both heavy ion and laser drivers. Research is being conducted in a coordinated manner by national laboratories, universities and industry. This paper provides an overview of U.S. research activities and discusses how interface considerations (such as beam propagation and target survival during injection) impact design choices.

1. Introduction

The U.S. has a significant R&D effort to develop the science and technology needed for inertial fusion energy (IFE). Here we focus on work being carried out to address issues related to the fusion chamber, the interface between the chamber and the driver, target fabrication, and target injection.

2. Fusion Chambers

Chamber research for heavy-ion-driven IFE is coordinated for the Department of Energy by the Virtual Laboratory for Technology (VLT). Currently, work is focused on a thick-liquid-wall chamber concept, HYLIFE-II [1]. This chamber uses jets of flibe (F, Li, Be molten salt) or flinabe (F, Li, Na, Be molten salt) to protect structures from target emissions including high-energy neutrons. The key issues being addressed deal with forming the liquid jets that make up the protective blanket and re-establishing the protective configuration between pulses (including clearing drops that could interfere with beam propagation or target injection). Several university experiments are addressing these issues. At UCB, experiments with water jets have demonstrated 1) the ability to produce very high quality (low surface ripple) cylindrical jets that are needed for the beam port region, 2) repetitive disruption (by chemical detonation) of an array of 96 jets [2], and 3) formation of vortex flow that can be used to protect the inner surface of beam pipes in the region of the final focus magnets.

Experiments at GT are characterizing surface ripple for various nozzle designs and flow conditions in the turbulent liquid sheets proposed for the protective blanket (see Fig. 1) [3]. GT is also conducting experiments and modeling on wetted wall chambers that use a thin film of liquid to absorb short ranged target emissions [4,5]. UCLA recently activated a plasmagun-based facility to study vaporization and condensation of flibe, including flowing liquid to enhance condensation [6]. LLNL is responsible for systems integration and modeling of the driver/chamber interface including radiation protection for the final focus magnets [7]. LLNL also conducts safety and environmental assessments including effects of target materials that enter the flibe. The ARIES team recently carried out an evaluation of the thick-liquid-wall chamber as part of their work on IFE.

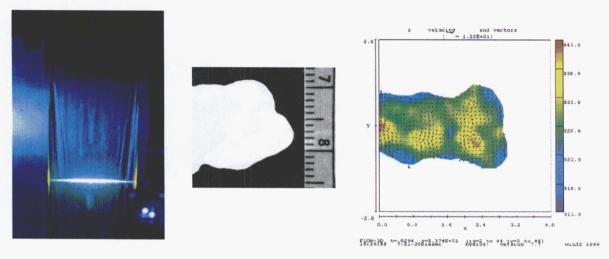


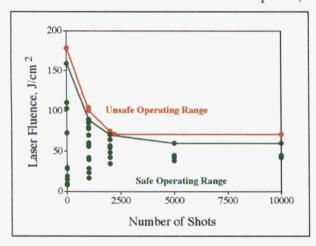
Fig. 1. Characterization of surface quality of turbulent $10 \text{ cm} \times 1 \text{ cm}$ water jet: Laser induced fluorescence diagnostic at GT (left), resulting image of edge region (center), 3D simulation carried out by UCLA (right).

Chamber research for laser-driven IFE is primarily conducted as part of the U.S. High Average Power Laser (HAPL) program complemented with VLT activities including a recent study of IFE by the ARIES team [8]. The HAPL program is a fully integrated research program that includes R&D on KrF lasers (NRL), diode-pumped solid-state lasers (LLNL), final optics materials (UCSD, LLNL), targets (design, fabrication, and injection), and chamber science and technology (UCSD, UW, LLNL). Chamber R&D has been focused on dry-wall designs that deal with the short-ranged target emissions (x-rays and debris) with either low-density gas fill or armor coating on the first wall structure. Survival of the first wall (from pulsed heating and radiation damage effects) for an acceptable time is a key issue for dry-wall chambers. An important consideration for the gas-protected concept is the effect of the gas on the cryogenic target during injection. Analysis indicates that there is a design window of target yield, chamber radius and gas pressure that will avoid first wall vaporization and permit target injection without damage to the fuel. An unresolved issue with this configuration is the long-term damage to the chamber wall from x-rays and ions. This is being investigated further, including the behavior of the wall material at IFE relevant temperatures, more detailed calculations of the ion and x-ray flux hitting the wall, and the possibility of using magnetic fields to divert the ions. A significant effort is being devoted to improve chamber dynamics models that predict the time evolution of post-shot chamber conditions (vaporization, condensation, aerosol formation) and to benchmark these models with experiments. Experiments and modeling of first wall response to pulse x-ray and debris heating have been carried out and more are planned.

3. Driver / Chamber Interface

The final focus magnets for heavy ion drivers are located as close to the chamber center as possible in order to achieve small beam spot sizes on the fusion target, which is necessary to achieve implosion symmetry and high target gain. The most recent HI driver design has 120 beams, with 60 coming from each side for the indirect-drive targets. The inner edge of the final focus magnet for each beam line is only 6 m from chamber center. The array of magnets must be closely packed in order to meet the geometric requirements of the target, which places constraints on the amount of neutron shielding that can be incorporated into the design. Work has been carried out to determine the shielding requirements for these magnets, and results of these detailed 3D neutronics calculations indicate that it is possible to provide enough shielding that the magnets will survive for the assumed 30 year life of the fusion power plant [7].

For laser driven IFE, the final optics can be located farther from chamber center (>20m), but they are in the direct line of sight of the target emissions. Experiments and analyses on radiation damage induced degradation of optical properties have been ongoing. Results to date indicate that a thin (1mm) transmissive grating made of fused silica will be acceptable for diode pumped solid-state lasers [9]. The color center growth saturates at an acceptable level. Most of the recent work on reflective optics (for use with krypton fluoride lasers) has been



with highly polished Al mirrors [10]. Figure 2 shows the allowable laser fluence as a function of cumulative pulses and indicates a baseline level that is well above the 5-10 J/cm² needed for IFE designs. It is likely that these will be implemented with a thin Al layer on a neutron damage resistant either reflective substrate. For transmissive final optics, damage from debris and x rays must be mitigated and several techniques to minimize these threats have been proposed (e.g., gas puffs and magnetic deflection).

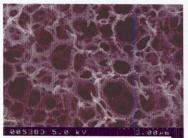
Fig. 2. Allowable laser fluence vs. number of pulses for pure Al mirror.

4. Target Fabrication and Injection

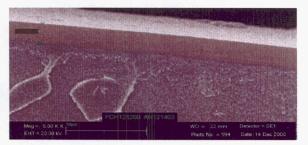
The target fabrication facility of an IFE power plant must supply about 500,000 targets per day, including manufacturing the spherical fuel capsule and other materials, filling the capsules with the DT fusion fuel, redistributing the frozen DT uniformly around the inside of the capsule (layering), and assembling the hohlraum (for indirect drive). Target fabrication work has concentrated on investigating and developing the various materials needed by the target designs and on fabrication techniques that could eventually scale to low cost and high production rate. In the area of materials, very low density foams doped with high-z materials (Fig. 3), which make up the energy deposition material in the heavy-ion driven target hohlraum, have been developed. Development of additional foam fabrication methods is underway, including laser-assisted chemical vapor deposition (LCVD), which may allow growth of micron-scale controlled structures for *in-situ* fabrication of the hohlraum foam components [11]. A small-scale fluidized bed that is capable of coating mandrels with

relevant ablator materials (Fig. 3) has also been built. For direct-drive targets, alloys of high-z capsule coatings are being optimized to give the desired properties of high reflectivity (for survival of the direct-drive cryogenic target in the high temperature chamber) and high permeation (for permeation filling with hydrogen isotopes). Models of material responses during the permeation filling step, and models for tritium inventory in the target facility have been developed and used to project acceptable quantities of tritium for plant operation [12].

The target is injected into the target chamber at a rate of 5–10 Hz. The DT layer must survive the exposure to the extremely rapid heat flux and remain highly symmetric, have a smooth inner ice surface finish, and reach the chamber center at a temperature of about 18.5 K [13]. Models of the thermo-mechanical effects on the advanced materials during injection have been developed. Fundamental measurements of the properties and response of DT under these unique conditions are being carried out. The target must be positioned at the center of the chamber with a placement accuracy of ± 5 mm and an alignment of the beams and the target of $\pm 20~\mu m$ or $\pm 200~\mu m$ for direct drive and indirect drive, respectively. An experimental injection and tracking system is being constructed to develop technologies and to demonstrate meeting these challenging requirements.



2% Au doped foam, 30 mg/cm³



~3 µm thick polymer coating deposited on a mandrel using fluidized bed technology

Fig. 3. Example of metal-doped foam for heavy ion target (left) and a spherical capsule with a polymer coating produced in a fluidized bed (right).

We must also understand the issues that affect low-cost target production – studies have consistently shown that a cost reduction of at least four orders of magnitude from current technologies will be needed for future electricity production. We have performed a cost analysis of manufacturing the direct drive target in a commercial process plant environment. This modeling of the process includes process flows, mass-energy balances, plant utilities, raw materials, quality control, waste handling and recycling, capital equipment cost amortization, and staffing requirements. Preliminary inputs to the model indicate that the cost goals of less than about \$0.25 per target can be readily met for the direct drive target. For indirect drive, more detailed hohlraum requirements, cheaper materials and easier fabrication processes are still being evaluated. A recent study of the costs for manufacturing the shell, then filling and layering resulted in an estimate of about \$0.11 each [14]. This is very encouraging because it leaves significant margin for the manufacture of the unique hohlraum components and assembly of the hohlraum with the shells. Evaluation of the cost of manufacturing the hohlraum itself is still underway.

Overall, the target technology program is addressing unique science and materials issues and is providing feedback from target fabricators to target designers as an essential element of the IFE program.

Conclusion

A research program to develop the science and technology of high pulse repetition rate chambers, target fabrication and target injection systems is in place and has made significant progress in recent years. Success in these areas, coupled with the continued development of efficient, high rep-rate drivers (lasers, heavy-ion accelerators and z-pinches) and the future demonstration of ignition on the National Ignition Facility (NIF) are critical to the development of IFE.

References

- [1] MOIR, R.W., et al., "HYLIFE-II: A molten-salt inertial fusion energy power plant design--final report", Fusion Technol. 25 (1994) 5.
- [2] PEMBERTON, S., JANTZEN, C., KUHN, J., and PETERSON, P., "Partial-pocket experiments for IFE thick-liquid pocket disruption and clearing", Fusion Technol. 39 (2001) 726.
- [3] REPERANT, J.J.R., DURBIN, S.G., YODA, M., ABDEL-KHALIK, S.I, and SADOWSKI, D.L., "Studies of turbulent liquid sheets for protecting IFE reactor chamber first walls", to appear in Fusion Eng. and Design (2002).
- [4] ANDERSON, J.K., DURBIN, S.G., SADOWSKI, D.L., YODA, M. and ABDEL-KHALIK, S.I., "Experimental studies of high-speed liquid films on downward facing surfaces", to appear in Fusion Sci. and Technol. (2002).
- [5] SHIN, S., ABDELALL, F.F., ABDEL-KHALIK, S.I., YODA, M. and SADOWSKI, D.L., "Fluid dynamic aspects of the porous wetted wall protection scheme for IFE reactors", submitted to Fusion Sci. and Technol. (2002).
- [6] CALDERONI, P., YING, A., SKETCHELY, T., and ABDOU, M., "Description of a facility for vapor clearing rates studies of IFE reactors flibe liquid chambers", Fusion Technol. 39 (2001) 711.
- [7] LATKOWSKI, J.F. and MEIER, W.R., "Heavy ion fusion final focus magnet shielding designs", Fusion Technol. **39** (2001) 798.
- [8] NAJMABADI, F., and the ARIES Team, "Assessment of chamber concepts for inertial fusion energy fusion power plants The ARIES-IFE study", (Proc. Inertial Fusion Sciences and Applications 2001) Elsevier (2002) 701-705.
- [9] LATKOWSKI, J.F., KUBOTA, A., CATURLA, M.J., DIXIT, S.N., SPETH, J.A and PAYNE, S., "Fused silica final optics for inertial fusion energy: radiation studies and system-level analysis", accepted for publication in Fusion Sci. & Technol.
- [10] TILLACK, M.S., PAYNE, S.A. and GHONIEM, N.M., "Damage threats and response of final optics for laser-fusion power plants", (Proc. Inertial Fusion Science and Applications 2001) Elsevier (2002) 717-721.
- [11] MAXWELL, J., RODRIGUEZ, L. and NOBILE, A., "Synthesis of high z low density foams for heavy ion fusion hohlraums", (2nd IAEA Technical Meeting on Physics and Technology of Inertial Fusion Energy Targets and Chambers, San Diego, CA, June 17-19, 2002) http://web.gat.com/conferences/iaea-tm/main.html.
- [12] SCHWENDT, A.M., et al., "Tritium inventory of inertial fusion energy target fabrication facilities: effect of foam density and consideration of target yield of direct drive targets", to appear in Fusion Sci. and Technol., MS No. 267702.
- [13] GOODIN, D.T., et al., "Developing target injection and tracking for inertial fusion energy power plants", Nuclear Fusion, 41, No. 5, 527.
- [14] GOODIN, D.T., et al., "A credible pathway for heavy ion driven target fabrication and injection", to be published in Lasers and Particle Beams, **20**, No. 4.